

APPENDIX

Given below are the titles, authors, and author affiliations of the 24 poster presentations.

Title	Author(s)	Affiliation
Corrosion of Fuel Channels at Extended Fuel Burnups	Y. Kuroda	Japan Atomic Power Company, Tokyo, Japan
Effects of Hydrides on the Ductile-Brittle Transition in Stress-Relieved, Recrystallized and Beta-State Zircaloy-4	J. Bai and C. Prioul J. Pelchat and F. Barcelo	Ecole Centrale de Paris, France CEN, Saclay, France
Cladding Hydrogen Uptake and Fuel Cladding Interaction	S. Yamanaka	Osaka University, Japan
The Development and Validation of a Tensile Test on Zircaloy Annular Ring Specimens	P. Bouffieux and D. Baron	EDF, France
Magnetic Study of Zircaloy II and Zircaloy with Niobium	C. Miyake and K. Gotoh	Osaka University, Japan
Effect of Refractory Element Addition on Nodular Corrosion Resistance of Zircaloy	H. Anada, Y. Shida, and T. Kodama	Sumitomo Metal Industries, Japan
The Operation Experience with the Zirconium-2.5Nb Alloy Technological Channels of the RBMK-1000 Reactors	P. A. Platonov, A. P. Eperin, I. A. Frolov, and A. V. Rjazzantzeva	Kurchatov Institute of Atomic Energy, Moscow, USSR
In-Pile Investigations of Zircaloy Cladding Performance at Extended Burnup	T. Karlsen, E. Kolstad, and C. Vitanza	OECD Halden Reactor Project, Halden, Norway
Separation of Hafnium from Zirconium-Tetrachloride by Molten Salt Electrolysis	T. Kirihaara and H. Ikeda I. Nakagawa Y. Seki	Mitsubishi Nucl. Fuel, Government Industrial Inst., Nagoya Mitsubishi Metal Corp.

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Irradiation and Post-Irradiation Examination of Heat Treated Zr-2.5Nb Pressure Tubes	M. H. Koike, T. Asada, S. Taniyama, and Y. Abe	Power Reactor and Nuclear Fuel Development Corp., Ibaraki, Japan
Potential Degradation Mechanisms of Spent Fuel Zirconium Alloy Cladding in a Deep Geological HLW Repository Environment	H. K. Manaktala	Southwest Research Institute, San Antonio, TX
A Study of Zircaloy Tubing with Compressive Hoop Stress	M. Megata	Nuclear Fuel Industries, Ibaraki, Japan
Fourier Transform Infrared (FTIR) Interferometry for Corrosion Film Thickness Measurements of Zirconium Alloys	N. Ramasubramanian	AECL, Ontario, Canada
Production of Zirconium-Alloy Fuel Cladding Tubes in the Soviet Union	F. G. Reshetnikov and A. V. Nikulina	Scient. Res. Institute of Inorganic Materials, Moscow, USSR
Fundamental Study on Welding of Zirconium	K. Suzuki	Japan Nuclear Fuel Service, Ltd., Tokyo, Japan
	T. Shige, O. Nakaya, and M. Hatano	Mitsubishi Heavy Industries, Ltd., Tokyo, Japan
Calorimetric and Resistivity Studies of Hydrides in Zr-2.5Nb Pressure Tube Material	K. Tashiro	Ontario Hydro Res. Div., Toronto, Canada
Fabrication Process for Hafnium Tubing for Control Rods	C. T. Wang and B. A. McClanahan	TWCA, Albany, OR
	H. Abe	Sumitomo Metal Industries, Tokyo, Japan
In-reactor Mechanical Behavior of BWR Fuel Channels	A. Jonsson	OKG AB, Figeholm, Sweden
	L. Hallstadius and U. Sundstrom	ABB Atom AB, Vasteras, Sweden

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Study of Zirconium/Stainless Steel Interface in Tubular Junctions Made by Eutectic Diffusion	J. Y. Blanc, R. LeGoff, and Ph. Regnier	Comm. a l'Energie Atomique, Gif Sur Yvette, France
Irradiation Performance of Advanced Zirconium Fuel Rods in a Commercial PWR	L. H. Boman, G. R. Kilp, M. G. Balfour, and G. Schoenberger	Westinghouse, Pittsburgh, PA
Investigation on the Corrosion Resistance of Zirconium in Nitric Acid	P. Fauvet, P. Mur, and J. Sannier	Comm. a l'Energie Atomique, Fontenay-aux-Roses, France
Corrosion Behavior of Zry-4 in Lithiated-Boricated Water at 360°C	J. M. Gras and D. Noel	Electricite de France, Moret/Loing, France
Second-Rank Tensor Properties in Hexagonal Polycrystals Thermal Expansion and Irradiation Growth of Textured Zircaloy Products	M. Humbert and C. Esling J. Diz and J. Bard	ISGMP, Metz, France Fragema, Lyon, France
Effects of Irradiation on the Microstructural Evolution and Corrosion Resistance of Zirconium Alloys	J. J. Kai, C. H. Tsai, J. J. Shiao, W. F. Hsieh, and C. S. Tu	National Tsing Hua University, Hsinchu, Taiwan